



MELCOR Applications and Development

Support Risk-Informing Regulations and Address Operating Reactor Issues

Severe accident research activities

- Retain experts who have severe accident phenomenological knowledge and maintain validated analytical tools
- International Collaboration
 - U.S. NRC Cooperative Severe Accident Research Program (CSARP)
 - Annual MELCOR Meetings
 - MELCOR Code Assessment Program (MCAP) (Fall 2016 in the United States)
 - European MELCOR User Group (EMUG) (Spring 2016 in London)
 - Asian MELCOR User Group (AMUG) (held October 2015 in Japan)
 - NEA/CSNI and European Commission



Model severe accidents and provide reasonable prediction of accident progression, source term,

MELCOR development

- Model containment thermal-hydraulic phenomena for design-basis analysis (DBA).
- Properly scale phenomena important to DBA and severe accidents from separate effect tests and integral effect tests to full size reactors.
- Develop models consistent with lumped parameter code framework (simplified vs. complex).

Targeted Applications

Design Objectives

- Perform plant-specific integrated analysis under postulated beyond DBA events and application to
- Perform containment response analysis under postulated DBA and beyond-DBA events.
- Perform accident analysis of nonreactor systems (e.g., spent fuel pool).

Success Criteria

- Prediction of phenomena in qualitative agreement with current understanding of physics and uncertainties are in quantitative agreement with experiments.
- Focus on mechanistic models where feasible with adequate flexibility for parametric models. Code is portable, robust, and relatively fast running, and the code maintenance follows
- established software quality assurance standards.
- Ensure availability of detailed code documentation (including user guide, model reference, and

Code development and regulatory applications State-of-the-Art Reactor secondary side onsequence Analysis (SOARCA) ARTISTII Site Level 3 PRA (Switzerland) Integral and separate uel Pool Study (NUREG-2161) ier 3 expedited fuel transfer release and chemistry (COMSECY-13-0030) PHEBUS FP/ISTP ISFSI rulemaking Molten core concrete Fukushima accident forensic nteraction and coolability analysis and reconstruction (DOE/NRC; BSAF[NEA]) **OECD-MCCI2 &** NRC/IRSN/EdF Fukushima NTTF 5.1, 5.2, 6.0 (United States) I: Filtered containment venting lodine chemistry and (BWR Mark I & II) behavior in containment OECD-BIP (Canada) OECD-STEM FP release & behavior Severe accident-induced SGTR (France) (VERCORS, France) uench of severely damaged fuel (KIT, Germany) evised source term (HBU/ MOX) CSARP/MCAP irconium fire initiation & lew and advanced reactor desig propagation Zirconium fire (United States)







